

NEYTRONLARNING MODDA BILAN TA'SIRI

FOR: RADIATION PHYSICS

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The neutron

n

$$I(J^P) = \frac{1}{2}(\frac{1}{2}^+)$$

Mass $m = 1.0086649160 \pm 0.0000000005$ u

Mass $m = 939.5654205 \pm 0.0000005$ MeV [a]

$$(m_n - m_{\bar{n}}) / m_n = (9 \pm 6) \times 10^{-5}$$

$$m_n - m_p = 1.2933324 \pm 0.0000005 \text{ MeV} \\ = 0.00138844919(45) \text{ u}$$

Mean life $\tau = 878.4 \pm 0.5$ s (S = 1.8)

$$c\tau = 2.6335 \times 10^8 \text{ km}$$

Magnetic moment $\mu = -1.9130427 \pm 0.0000005 \mu_N$

Electric dipole moment $d < 0.18 \times 10^{-25}$ e cm, CL = 90%

Mean-square charge radius $\langle r_n^2 \rangle = -0.1155 \pm 0.0017 \text{ fm}^2$

Magnetic radius $\sqrt{\langle r_M^2 \rangle} = 0.864^{+0.009}_{-0.008} \text{ fm}$

Electric polarizability $\alpha = (11.8 \pm 1.1) \times 10^{-4} \text{ fm}^3$

Magnetic polarizability $\beta = (3.7 \pm 1.2) \times 10^{-4} \text{ fm}^3$

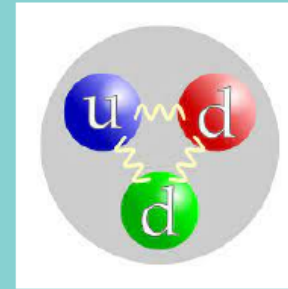
Charge $q = (-0.2 \pm 0.8) \times 10^{-21} e$

Mean $n\bar{n}$ -oscillation time $> 8.6 \times 10^7$ s, CL = 90% (free n)

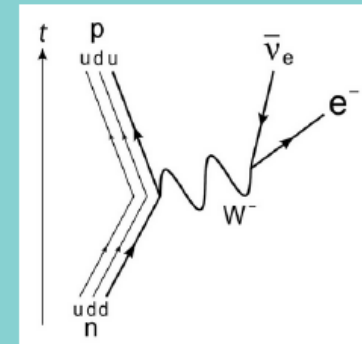
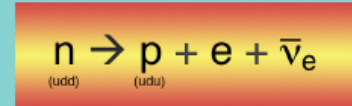
Mean $n\bar{n}$ -oscillation time $> 4.7 \times 10^8$ s, CL = 90% [g] (bound n)

Mean nn' -oscillation time > 448 s, CL = 90% [h]

$$M = 1 \\ Q = 0$$



$$T_{1/2} = 10.14 \text{ min}$$



NEYTRONLAR KLASSIFIKATSIYA

1) High Energy neutrons:

$$E_n > 100 \text{ MeV}$$

2) Fast Neutrons:

$$100 \text{ keV} < E_n < 10 \text{ MeV}$$

3) Epithermal Neutrons:

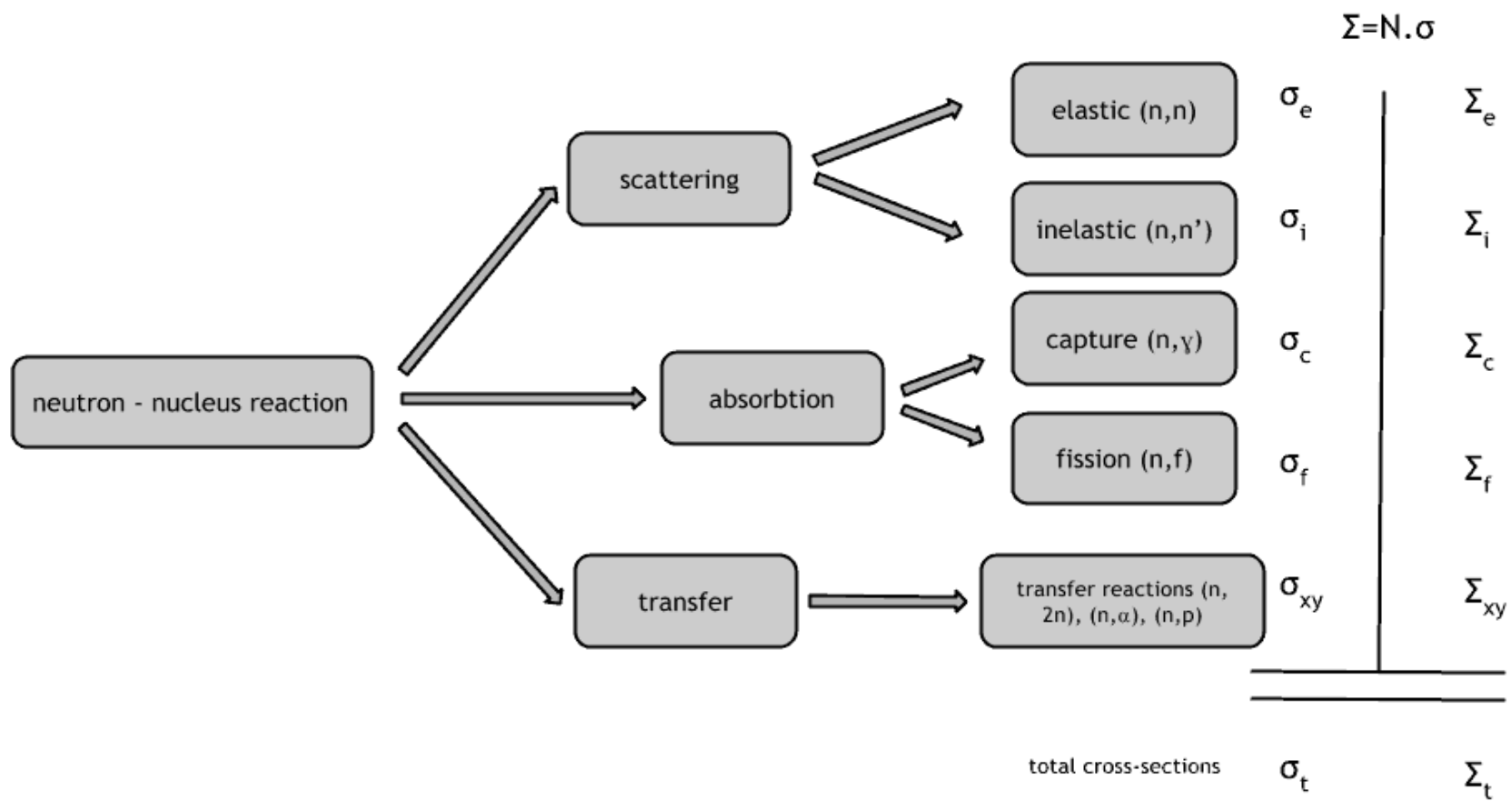
$$0.1 \text{ eV} < E_n < 100 \text{ keV}$$

4) Thermal/Slow Neutron:

$$E_n = 1/40 \text{ eV}$$

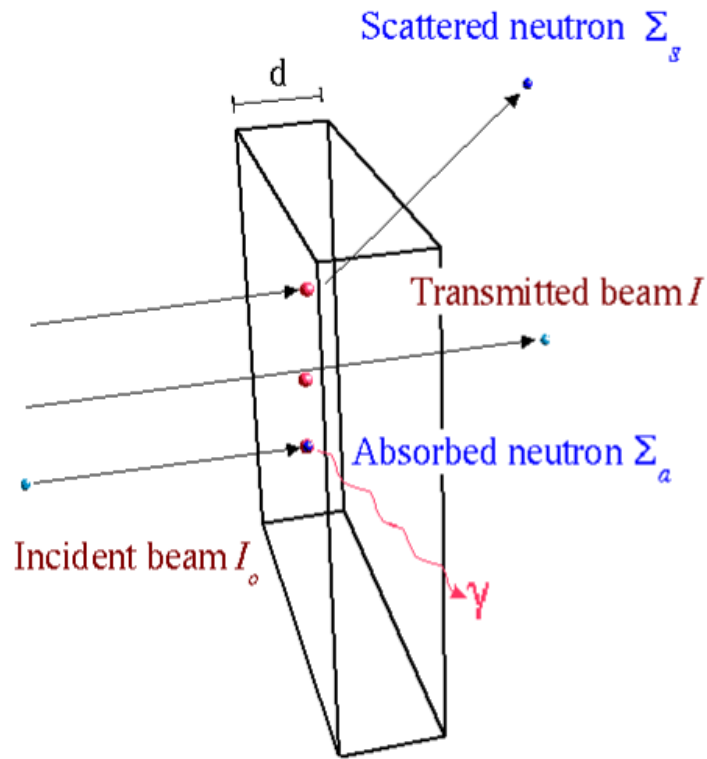
5) Cold/Ultracold Neutrons:

$$E_n < \text{meV} \dots \mu\text{eV}$$

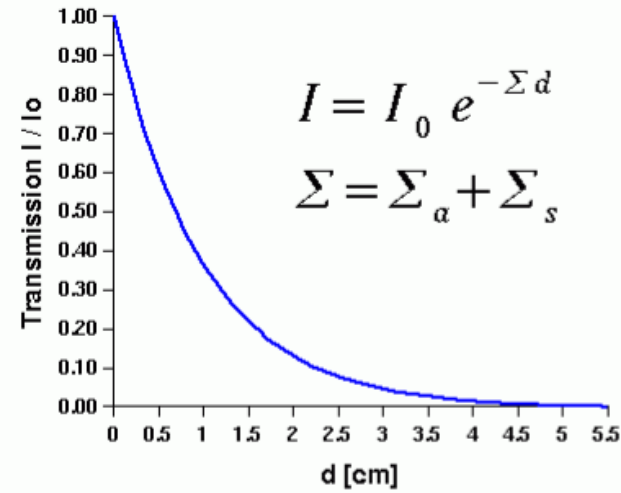


Narrow Beam Attenuation

$$I = I_0 e^{-\Sigma d}$$



Exponential Attenuation Law



Macroscopic Cross Section Σ

$$\Sigma = N \sigma \quad [\text{cm}^{-1}]$$

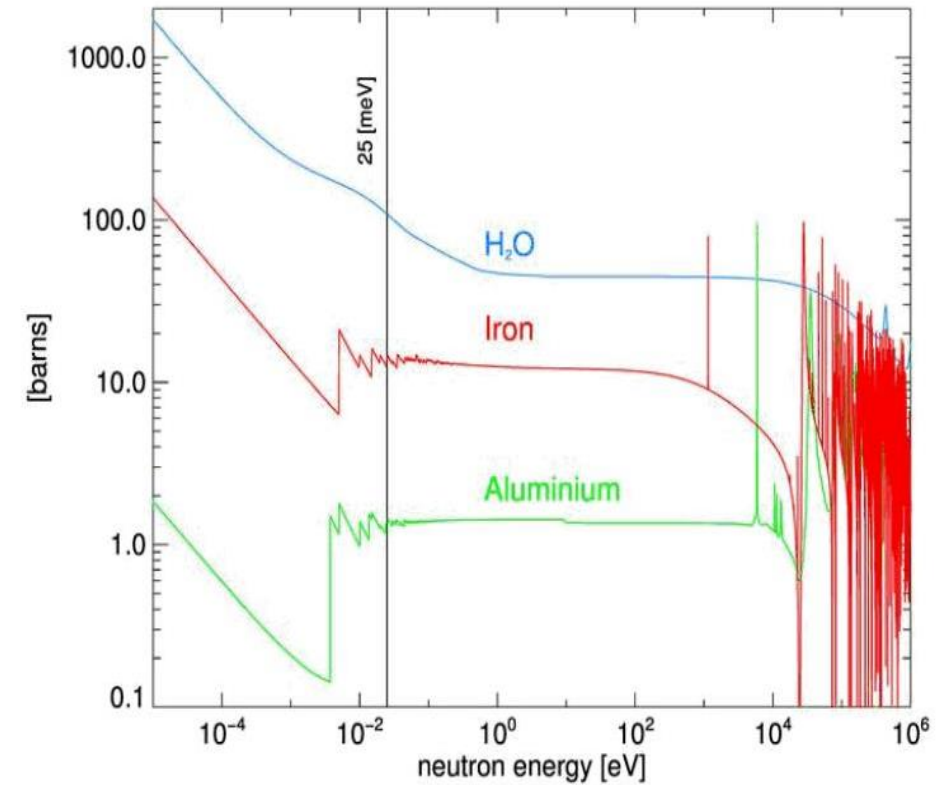
$$N = \frac{\rho}{A} N_A \quad [\text{cm}^{-3}]$$

N := number density $[\text{cm}^{-3}]$

ρ := material density $[\text{g cm}^{-3}]$

A := atomic weight $[\text{g mol}^{-1}]$

N_A := Avogadro number $6.022 \cdot 10^{23} [\text{mol}^{-1}]$



Thermal neutron cross section at 0.025 eV

Material	ρ	σ [barns]	Σ [1/cm]
Water	1	112	3.74
Iron	7.9	14	1.18
Aluminium	2.7	1.7	0.1

Common reactions used for neutron detection at low (and high) energies:

Elastic scattering:

- $n + {}^1\text{H} \rightarrow n + {}^1\text{H}$
- $n + {}^2\text{H} \rightarrow n + {}^2\text{H}$ (abund.=0.015%)

Charged particle:

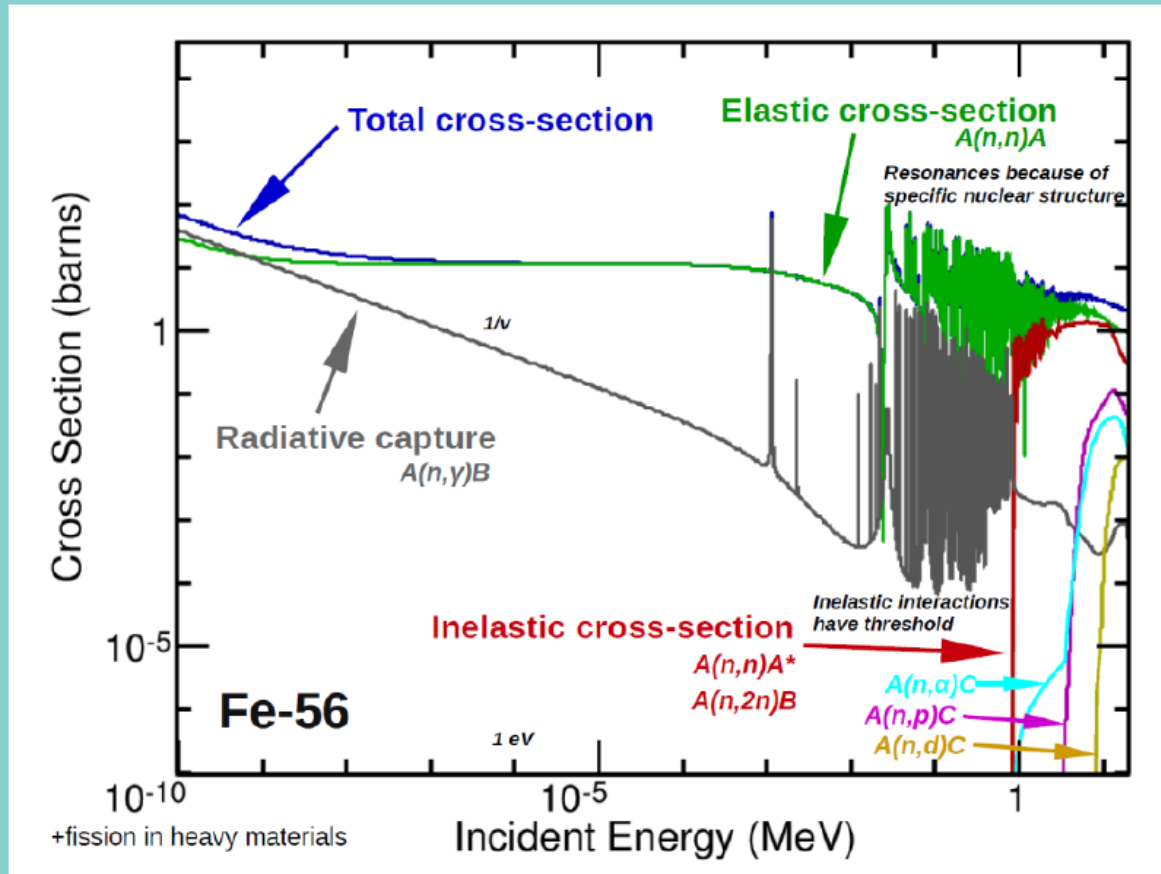
- $n + {}^3\text{He} \rightarrow {}^3\text{H} + {}^1\text{H} + 0.764 \text{ MeV}$ (abund.=0.00014%)
- $n + {}^6\text{Li} \rightarrow {}^4\text{He} + {}^3\text{H} + 4.79 \text{ MeV}$ (abund.=7.5%)
- $n + {}^{10}\text{B} \rightarrow {}^7\text{Li}^* + {}^4\text{He} \rightarrow {}^7\text{Li} + {}^4\text{He} + 0.48 \text{ MeV } \gamma + 2.3 \text{ MeV}$
(abund.=19.9%,
b.r.=93%)

Radiative capture:

- $n + {}^{155}\text{Gd} \rightarrow {}^{156}\text{Gd}^* \rightarrow \gamma\text{-ray} + \text{CE spectrum}$ (abund.=14.8%)
- $n + {}^{157}\text{Gd} \rightarrow {}^{158}\text{Gd}^* \rightarrow \gamma\text{-ray} + \text{CE spectrum}$ (abund.=15.7%)

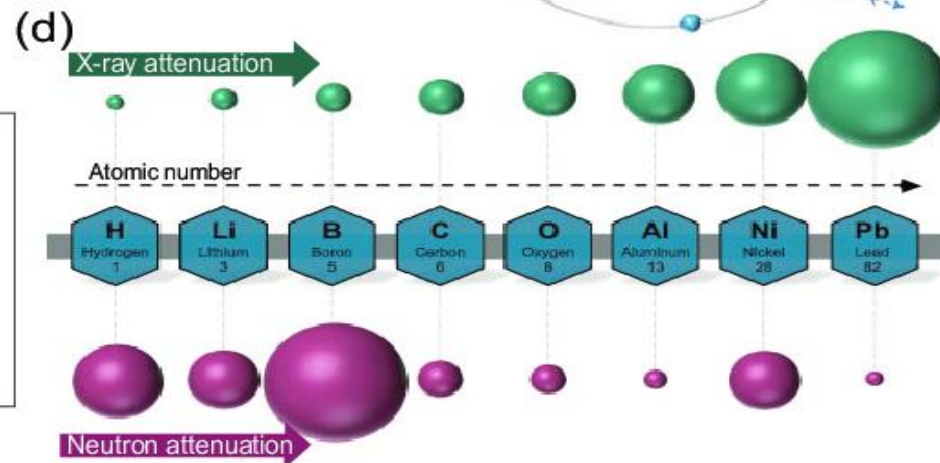
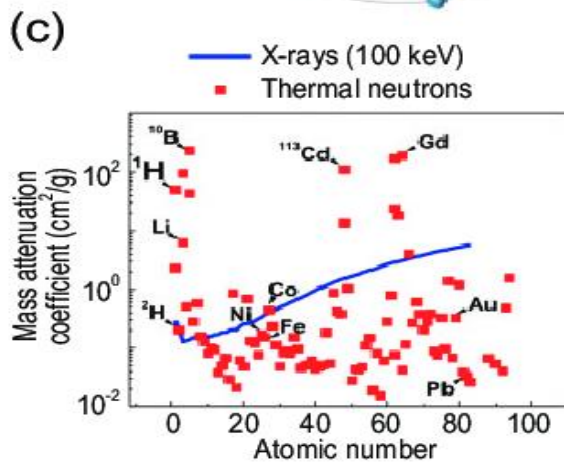
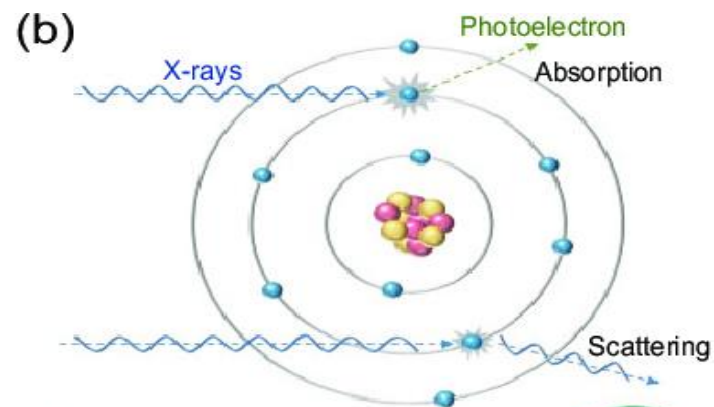
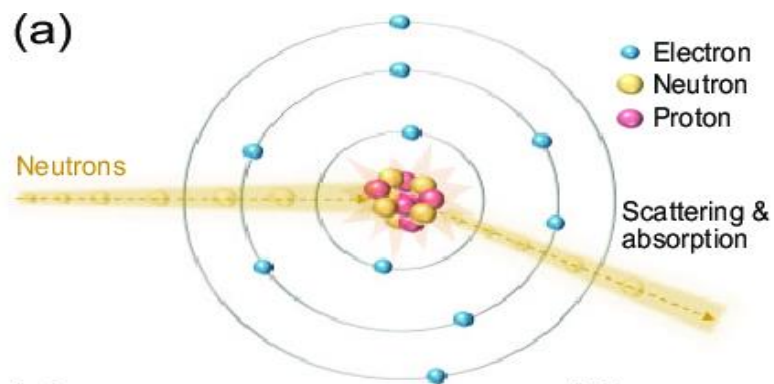
Fission:

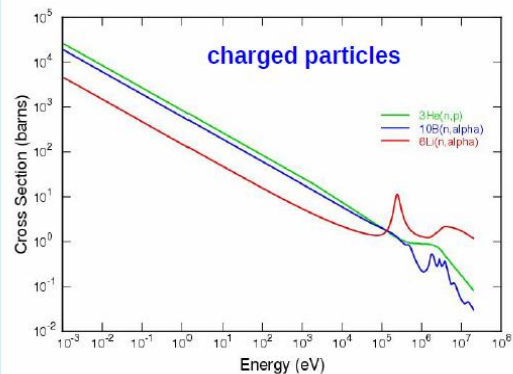
- $n + {}^{235}\text{U} \rightarrow \text{fission fragments} + \sim 160 \text{ MeV}$
- $n + {}^{239}\text{Pu} \rightarrow \text{fission fragments} + \sim 160 \text{ MeV}$
- $n + {}^{238}\text{U} \rightarrow \text{fission fragments} + \sim 160 \text{ MeV}$



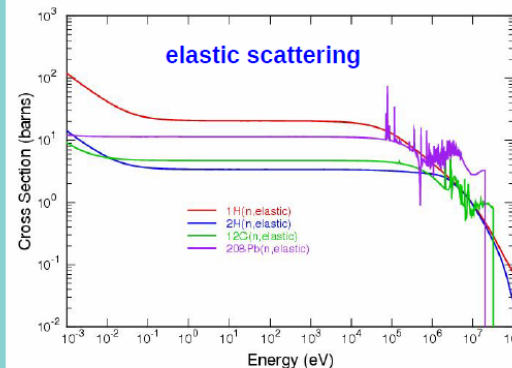
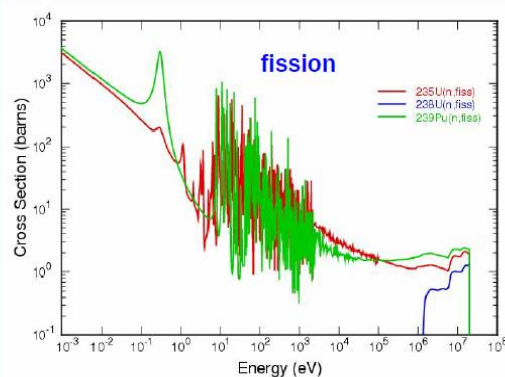
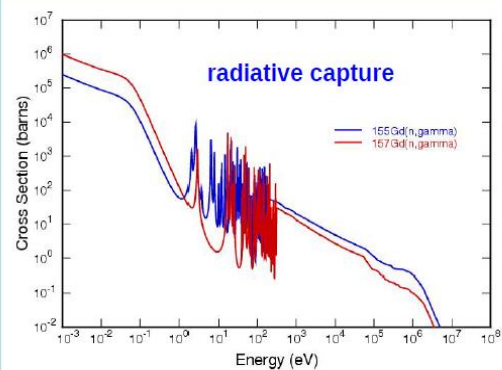
$$\sigma_{tot} = \sigma_{n,n} + \sigma_{n,\gamma} + \sigma_{n,n'} + \sigma_{n,\alpha} + \sigma_{n,p} + \dots$$

Elastic scattering and radiative capture channels are always open

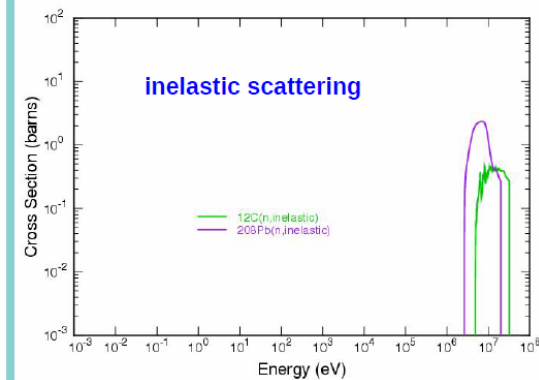
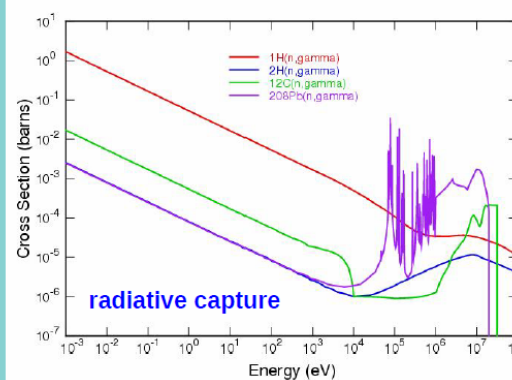




**Cross section
energy dependence
of important
reactions**



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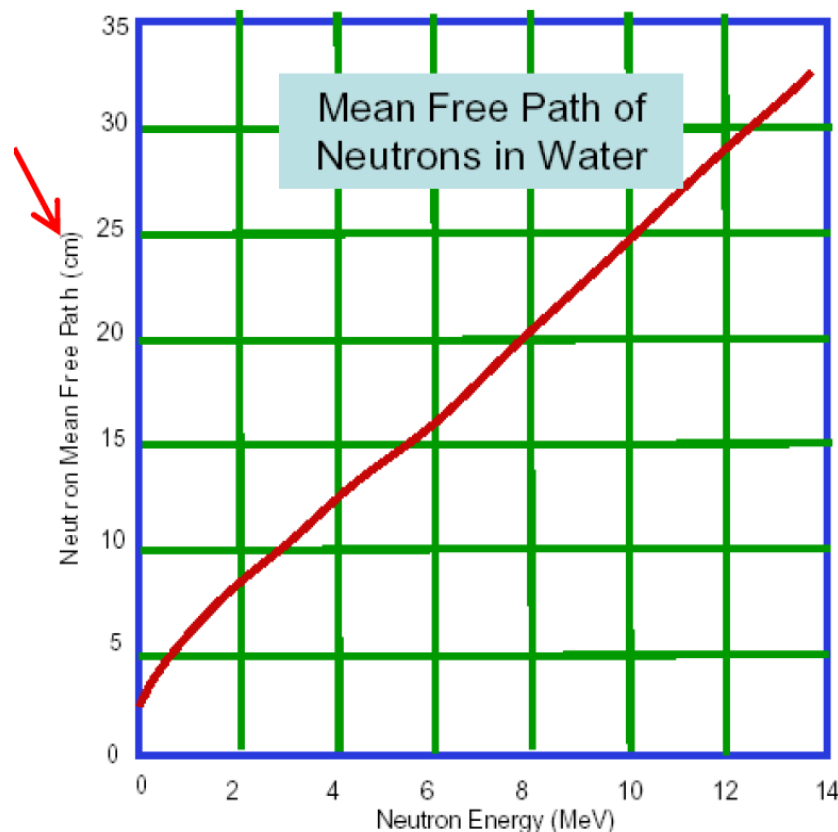


► Neutroning O'rtacha Erkin Yo'li

- O'rtacha Erkin Yo'l: Neutronlarning keyingi to'qnashuvlari orasidagi o'rtacha masofa
- $N(x) = N(0)e^{(-x/\lambda)}$ - Bu formula orqali neutroning o'rtacha erkin yo'li hisoblanadi.
- $\lambda = 1/\mu = 1/(\rho\sigma)$ - O'rtacha yo'l uzunligi.
 - ρ : atom zichligi
 - σ : kesishuv maydoni
 - λ : o'rtacha yo'l uzunligi

Average distance between successive collisions

Neutron Mean Free Path



$$N(x) = N(0)e^{-x/\lambda}$$

$$\lambda = \frac{1}{\mu} = \frac{1}{\rho\sigma} \quad mfp$$

ρ : atomic density
(# atoms/volume)
 σ : cross section

λ = average path length in medium between 2 collisions

$$\sigma = \sigma_{TOT} = \sigma_{ELASTIC} + \sigma_{INELASTIC} + \sigma_{CAPTURE} + \dots$$

$$\Gamma_{FWHM} = 2.35 \cdot \sigma$$

THANK YOU!